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Special and Unique: Lessons Learned from the Implementation of Pull-Off Testing as Part of Project Quality Assurance and Quality Control at Operating Nuclear Power Plants



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## Agenda

- America's Aging Nuclear Fleet
- U.S. NRC Quality Requirements for Licensees
- What is Pull-Off Testing?
- Pull-Off Testing Acceptance Criteria
- Lessons Learned
- Questions?



## **America's Aging Nuclear Fleet**

U.S. Operating Commercial Nuclear Power Reactors



- America's operating nuclear generation fleet is responsible for ~20% of the United States' electrical use
- U.S. NRC regulates the operation of (94) licensed commercial nuclear power reactors



## **America's Aging Nuclear Fleet**

License Renewals Granted for Operating Nuclear Power Reactors



▲ Original License (8) ▲ License Renewal Granted (82) ▲ Subsequent License Renewal Granted (4)

Note: The NRC has issued a total of 96 license remeands; 8 of these units have permanently shut down. Data are see of August 2020. For the most recent information, go to the Dataset Index Web page at https://www.nrc.gov/reading-mr/doc-collective\_s/data.ets/.

From U.S. NRC 2020-2021 Information Digest

- Each reactor is initially licensed for 40 years
- U.S. NRC has granted (86) of the (94) licensed commercial nuclear power reactors 20-year license extensions
  - (4) have been granted a 2<sup>nd</sup>
    20-year license extension





## **America's Aging Nuclear Fleet**

U.S. Commercial Nuclear Power Reactors —Years of Operation by the End of 2020



Note: Ages are based on operating license issued date and have been rounded up to the end of the year. For the most recent information, go to the Dataset Index Web page at https://www.nrc.gov/reading-rm/ doc-collections/datasets/.





From U.S. NRC 2020-2021 Information Digest

## **NRC Quality Requirements for Licensees**

Appendix B to 10 CFR Part 50 establishes the Quality Assurance Criteria for Nuclear Power Plants and states:

"Nuclear power plants and fuel reprocessing plants include structures, systems, and components that prevent or mitigate the consequences of postulated accidents that could cause undue risk to the health and safety of the public. This appendix establishes quality assurance requirements for the design, manufacture, construction, and operation of those structures, systems, and components. The pertinent requirements of this appendix <u>apply to all</u> <u>activities</u> affecting the safety-related functions of those structures, systems, and components; these activities include <u>designing, purchasing</u>, fabricating, handling, <u>shipping, storing</u>, cleaning, <u>erecting, installing, inspecting,</u> <u>testing,</u> operating, maintaining, <u>repairing,</u> refueling, and modifying."



### **NRC Quality Requirements for Licensees**

Typical Set of Project Documents for a project at a Nuclear Power Plant and the responsible party

- Engineering Change Package (Utility)
- Plant Procedures (Utility)
- Work Order (Utility)
- Project Specifications / Criteria Documents (Utility/Contractor)
- Quality Plan (Utility/Contractor)
- Project Execution Plan (Contractor)
- Inspection and Test Plans (Contractor)



### How does one successfully perform pulloff testing as part of a project quality plan at a nuclear power plant?



## What is Pull-Off Testing?



*Fig. 1.1: Components of a composite overlay/ repair system* 



*Fig. 3.3: Specimen preparation and test setup (composite system)* 



Figures from ICRI Technical Guideline No. 210.3R-2013

## **Acceptance Criteria and Test Plans**

- No generally accepted pull-off test criteria
- Criteria should be determined based on experience
- Concrete is a variable material
- Results can be reviewed and evaluated by a competent individual to determine if acceptable



## **Acceptance Criteria and Test Plans**

In nuclear power industry, acceptance criteria, in general, are:

- Specific
- Rigid
- Not Open for Interpretation

Testing and acceptance criteria need to be well documented and vetted prior to being put in use via:

- Documented performer training and qualifications
- Written test procedures
- Written test plans







*Fig. 3.9.a: Concrete substrate test: Failure Mode S1* 

Nuclear Industry "error trap":

Work orders & specific sequence of tasks



Figure from ICRI Technical Guideline No. 210.3R-2013



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*Fig. 3.9.a: Concrete substrate test: Failure Mode S1* 

Figure from ICRI Technical Guideline No. 210.3R-2013

Core ID	Bond Strength (PSI)	Failure Mode		
NL7A2C1	72.9	S2		
NL7A2C2	70.9	S1		
NL7A2C3	175.2	S1		
NL7A2C4	91.6	S1		
NL7A2C5	88.9	S1		
NL8A3C1	47.1	S1		
NL8A3C2	263	S1		
NL8A3C3	143.1	S1		
NL8A3C4	216.9	S1		
NL8A3C5	183.9	S1		
NL9A2C1	185.7	S1		
NL9A2C3	30	S1		
NL9A2C4	200.8	S1		
NL9A2C6	102.8	S1		





Core ID	Bond Strength (PSI)	trength SI) Failure Mode			
NL7A2C1	72.9	S2			
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NL8A3C1	47.1	S1			
NL8A3C2	263	S1			
NL8A3C3	143.1	S1			
NL8A3C4	216.9	S1			
NL8A3C5	183.9	S1			
NL9A2C1	185.7	S1			
NL9A2C3	30	S1			
NL9A2C4	200.8	S1			
NL9A2C6	102.8	S1			





*Fig. 3.9.b: Concrete substrate test: Failure Mode S2* 

	NL	<u>.</u> 9A
Figure from ICRI Technical Guideline No. 210.3R-2013	3	

Core ID	Bond Strength (PSI)	Failure Mode			
NL7A2C1	72.9	S2			
NL7A2C2	70.9	S1			
NL7A2C3	175.2	S1			
NL7A2C4	91.6	S1			
NL7A2C5	88.9	S1			
NL8A3C1	47.1	S1			
NL8A3C2	263	S1			
NL8A3C3	143.1	S1			
NL8A3C4	216.9	S1			
NL8A3C5	183.9	S1			
NL9A2C1	185.7	S1			
NL9A2C3	30	S1			
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NL9A2C6	102.8	S1			





Core ID	Bond Strength (PSI)	Failure Mode			
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NL7A2C5	88.9	S1			
NL8A3C1	47.1	S1			
NL8A3C2	263	S1			
NL8A3C3	143.1	S1			
NL8A3C4	216.9	S1			
NL8A3C5	183.9	S1			
NL9A2C1	185.7	S1			
NL9A2C3	30	S1			
NL9A2C4	200.8	S1			
NL9A2C6	102.8	S1			



#### How did this happen:

- Lack of understanding of project execution steps by external stakeholders
- Overconfidence of internal stakeholders that work order was identical to previous project on adjacent unit

#### Changes moving forward:

- Quality Plan enhancements: In-process quality check list for field leaders and quality engineer developed
- All project documents reviewed and updated to remove potential error traps/ambiguous language
- Work Order was revised to list all tasks in the proper order



## Lesson Learned #2: Depth of Core into Substrate

#### Imprecisely tracking & underestimating drill depth



Fig. 3.9.d: Composite repair test: Failure Mode R2



Figure from ICRI Technical Guideline No. 210.3R-2013

## Lesson Learned #2: Depth of Core into Substrate

#### How did this happen:

- Core depths were average over 1 ft x 1 ft grid along repair area.
- The testing lead reduced target drill depth from 1.5 inch to 1.0 inch

		QUANTITIES & DEMOLITION DEPTH (INCH)																		
	1	2	3	4	5	6	7	8	9	10	11	12	13	14	15	16	17	18	19	20
1	4.5	4	3.75	4.25	3.25	3.75	4.25	3.75	3.5	3	3	3.5	3.75	4.25	4	4.75	4.75	5.75	5.25	4.5
2	3.25	3.5	4	4	4.25	4.25	3.5	4.25	2.5	2.75	3.5	2.75	2.5	3.25	3.5	4	4.25	4.2	5	4.5
3	3															3.25	2.75	2.75	5.5	3.75
4																				



## Lesson Learned #2: Depth of Core into Substrate

#### Changes moving forward:

- Project specification document was updated.
- Pre locating Composite Pull-off test locations
- Project Quality meeting to review incident
- More rigorous pre-job brief

Impact to project: pull-off testing paused for 2 months!



Compressive strength was implied as a contributing cause to low pull-off test results

Core Id	Bond Strength (psi)	Failure Mode	Mode R5 and
1	151	R5	Mode R6=
2 <	143	R5	failure in the
3	127	R3	substrate,
4	229	R6	Mode R3=
5	302	R5	failure at the
6	175	R3	bond surface
7	350	R3	
8 <	92	R5	
9	Invalid	test result	
10	64	R5	>
11	207	R5	
12	63.6	R5	$\triangleright$
13	183	R5	
14	159	R5	

### **Definition: Operability**

 A system, subsystem, train, component, or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified safety function(s), and when all necessary attendant instrumentation, controls, normal or emergency electrical power, cooling and seal water, lubrication and other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its specified safety function(s) are also capable of performing their related support function(s).



From U.S. NRC Inspection Manual Chapter 0326 – Operability Determinations

#### How did this happen:

- Project specifications were not detailed enough to address unsatisfactory results
- Contractor did not understand the potential negative impact of an unsatisfactory result and could not properly explain the technical basis for the test and how to properly interpret the results
- Owner's Engineer did not understand the what the test results meant prior to accepting this test as part of the project quality plan and made conservative assumptions when unsure



#### Changes moving forward:

- Prepared a white paper was prepared outlining the entire pulloff test process
- Prepared a formal written procedure for performing pull-off tests.
- Retrained project team
- Prepared a "Pull-Off Test Validation and Implementation Plan"
  - Acceptance criteria was changed from "The minimum tensile bond strength for the repair concrete is 150 psi or failure within the substrate" to...





## **Questions?**

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